Nuclear Engineering and Design 142 (1993) 355–356
North-Holland

Author Index to Volume 142

Aoki, S., see Kikuchi, M. 142 (1993) 51
Arndt, E.G., see Naus, D.J. 142 (1993) 327
Arndt, E.G., see Oland, C.B. 142 (1993) 179

Bartholomé, G., W. Kastner and E. Keim, Design and calibration of leak detection systems by thermal hydraulics and fracture mechanics analyses 142 (1993) 1
Bhandari, S. and J.C. Leroux, Evaluation of crack opening times and leakage areas for longitudinal cracks in a pressurized pipe. Part I. Model for maximum leak areas and its validation 142 (1993) 15
Bhandari, S. and J.C. Leroux, Evaluation of crack opening times and leakage areas for longitudinal cracks in a pressurized pipe. Part II. Application of proposed model and fracture dynamics 142 (1993) 21
Brust, F.W., see Leis, B.N. 142 (1993) 69

Chen, J.T., Consideration of external events in the individual plant examination program 142 (1993) 231
Choi, J.B., see Kim, Y.J. 142 (1993) 61
Chung, H.D., see Kim, Y.J. 142 (1993) 61
Cooper, S.E., E.V. Lofgren, P.K. Samanta and S.-M. Wong, Dependent failure analysis of NPP data bases 142 (1993) 137

Doctor, S.R., see Vo, T.V. 142 (1993) 239
Dong, Y.M., W. Yang and K.C. Hwang, Elastic-plastic defect assessment based on ductile fracture process 142 (1993) 27
Drew, D.A., see Kalkach-Navarro, S. 142 (1993) 341

Ebisawa, K. and T. Uga, Evaluation methodology for seismic base isolation of nuclear equipments 142 (1993) 319
Ellingwood, B., see Naus, D.J. 142 (1993) 327
Ellingwood, B.R. and Y. Mori, Probabilistic methods for condition assessment and life prediction of concrete structures in nuclear power plants 142 (1993) 155

Flesch, B., see Pitner, P. 142 (1993) 89

Geidl, V., see Phillips, J.H. 142 (1993) 167
Gilman, J., see Nickell, R.E. 142 (1993) 77
Gore, B.F., see Vo, T.V. 142 (1993) 239
Granger, B., see Pitner, P. 142 (1993) 89

Hojo, K., M. Takenaka, H. Kaguchi, G. Yagawa and S. Yoshimura, Applications of probabilistic fracture mechanics to FBR components 142 (1993) 43
Hwang, K.C., see Dong, Y.M. 142 (1993) 27

Kaguchi, H., see Hojo, K. 142 (1993) 43
Kalkach-Navarro, S., R.T. Lahey, Jr., D.A. Drew and R. Meyder, Interfacial area density, mean radius and number density measurements in bubbly two-phase flow 142 (1993) 341
Kastner, W., see Bartholomé, G. 142 (1993) 1
Keim, E., see Bartholomé, G. 142 (1993) 1
Kikuchi, M., S. Yoshimura, S. Aoki, T. Nishioka, N. Nakagaki, N. Miyazaki, N. Mukaimachi, S. Sasaki, T. Shimakawa, T. Watanabe and G. Yagawa, Analysis of stable crack growth across welded fusion line: Japanese round-robin 142 (1993) 51
Kim, Y.J., S.H. Son, J.B. Choi and H.D. Chung, Development of elastic-plastic integrity evaluation system for pressure vessel and pipings 142 (1993) 61

Lahey, Jr., R.T., see Kalkach-Navarro, S. 142 (1993) 341
Lehner, J.R., C.C. Lin and P. Neogy, Identification and assessment of containment and release management strategies 142 (1993) 189
Leis, B.N. and F.W. Brust, Validation of room-temperature primary creep crack-growth analysis for surface-cracked pipes 142 (1993) 69
Leroux, J.C., see Bhandari, S. 142 (1993) 15
Leroux, J.C., see Bhandari, S. 142 (1993) 21
Lin, C.C., see Lehner, J.R. 142 (1993) 189
Liu, W.K., see Tsukimori, K. 142 (1993) 267
Liu, W.K., see Tsukimori, K. 142 (1993) 299
Lofgren, E.V., see Cooper, S.E. 142 (1993) 137

Magleby, H.L., see Phillips, J.H. 142 (1993) 167
Meyder, R., see Kalkach-Navarro, S. 142 (1993) 341
Miyazaki, N., see Kikuchi, M. 142 (1993) 51
Mori, Y., see Ellingwood, B.R. 142 (1993) 155

Mori, Y., see Naus, D.J. 142 (1993) 327
Mukaimachi, N., see Kikuchi, M. 142 (1993) 51

Nakagaki, N., see Kikuchi, M. 142 (1993) 51
Naus, D.J., C.B. Oland, B. Ellingwood, Y. Mori and E.G. Arndt, An overview of the ORNL/NRC program to address aging of concrete structures in nuclear power plants 142 (1993) 327
Naus, D.J., see Oland, C.B. 142 (1993) 179
Neogy, P., see Lehner, J.R. 142 (1993) 189
Nickell, R.E., D.F. Quiñones and J. Gilman, Evaluation of flawed piping under dynamic loading 142 (1993) 77
Nishioka, T., see Kikuchi, M. 142 (1993) 51

Oland, C.B., D.J. Naus and E.G. Arndt, Research and development at the Structural Materials Information Center 142 (1993) 179
Oland, C.B., see Naus, D.J. 142 (1993) 327

Pennell, W.E., Heavy-Section Steel Technology Program Overview 142 (1993) 117
Pennell, W.E., Heavy-Section Steel Technology Program: recent developments in crack initiation and arrest research 142 (1993) 255
Phillips, J.H., W.S. Roesener, H.L. Magleby and V. Geidl, Incorporation of passive components aging into PRAs 142 (1993) 167
Pitner, P., T. Riffard, B. Granger and B. Flesch, Application of probabilistic fracture mechanics to optimize the maintenance of PWR steam generator tubes 142 (1993) 89

Quiñones, D.F., see Nickell, R.E. 142 (1993) 77

Riffard, T., see Pitner, P. 142 (1993) 89
Roesener, W.S., see Phillips, J.H. 142 (1993) 167

Samanta, P.K., see Cooper, S.E. 142 (1993) 137
Sasaki, S., see Kikuchi, M. 142 (1993) 51
Shimakawa, T., see Kikuchi, M. 142 (1993) 51
Shotkin, L.M., U.S. NRC Accident Management Program 142 (1993) 225
Simonen, F.A., see Vo, T.V. 142 (1993) 239
Son, S.H., see Kim, Y.J. 142 (1993) 61

Takenaka, M., see Hojo, K. 142 (1993) 43
Tsao, Y.H., see Tsukimori, K. 142 (1993) 299
Tsukimori, K., W.K. Liu and R.A. Uras, Formulation of dynamic stability of fluid-filled shells 142 (1993) 267
Tsukimori, K., W.K. Liu, Y.H. Tsao and R.A. Uras, Dynamic stability characteristics of fluid-filled shells under multiple excitations 142 (1993) 299

Uga, T., see Ebisawa, K. 142 (1993) 319
Uras, R.A., see Tsukimori, K. 142 (1993) 267
Uras, R.A., see Tsukimori, K. 142 (1993) 299
Utsunomiya, T. and K. Watanabe, Fracture criterion of a mixed-mode crack based on CED (Crack Energy Density) 142 (1993) 101

Vo, T.V., B.F. Gore, F.A. Simonen and S.R. Doctor, Risk assessment in setting inservice inspection priorities at Light Water Reactors 142 (1993) 239

Watanabe, K., see Utsunomiya, T. 142 (1993) 101
Watanabe, T., see Kikuchi, M. 142 (1993) 51
Wesley, D.A., Interfacing Systems LOCA (ISLOCA) component pressure capacity methodology and typical plant results 142 (1993) 209
Wong, S.-M., see Cooper, S.E. 142 (1993) 137

Yagawa, G., see Hojo, K. 142 (1993) 43
Yagawa, G., see Kikuchi, M. 142 (1993) 51
Yang, W., see Dong, Y.M. 142 (1993) 27
Yoshimura, S., see Hojo, K. 142 (1993) 43
Yoshimura, S., see Kikuchi, M. 142 (1993) 51

Nuclear Engineering and Design 142 (1993) 357–358
North-Holland

Subject Index to Volume 142

Thermal and fluid design

Interfacial area density, mean radius and number density measurements in bubbly two-phase flow, S. Kalkach-Navarro, R.T. Lahey, Jr., D.A. Drew and R. Meyder 142 (1993) 341

Engineering mechanics

Evaluation of crack opening times and leakage areas for longitudinal cracks in a pressurized pipe. Part I. Model for maximum leak areas and its validation, S. Bhandari and J.C. Leroux 142 (1993) 15

Evaluation of crack opening times and leakage areas for longitudinal cracks in a pressurized pipe. Part II. Application of proposed model and fracture dynamics, S. Bhandari and J.C. Leroux 142 (1993) 21

Elastic-plastic defect assessment based on ductile fracture process, Y.M. Dong, W. Yang and K.C. Hwang 142 (1993) 27

Applications of probabilistic fracture mechanics to FBR components, K. Hojo, M. Takenaka, H. Kaguchi, G. Yagawa and S. Yoshimura 142 (1993) 43

Analysis of stable crack growth across welded fusion line: Japanese round-robin, M. Kikuchi, S. Yoshimura, S. Aoki, T. Nishioka, N. Nakagaki, N. Miyazaki, N. Mukaimachi, S. Sasaki, T. Shimakawa, T. Watanabe and G. Yagawa 142 (1993) 51

Development of elastic-plastic integrity evaluation system for pressure vessel and pipings, Y.J. Kim, S.H. Son, J.B. Choi and H.D. Chung 142 (1993) 61

Validation of room-temperature primary creep crack-growth analysis for surface-cracked pipes, B.N. Leis and F.W. Brust 142 (1993) 69

Evaluation of flawed piping under dynamic loading, R.E. Nickell, D.F. Quiñones and J. Gilman 142 (1993) 77

Application of probabilistic fracture mechanics to optimize the maintenance of PWR steam generator tubes, P. Pitner, T. Riffard, B. Granger and B. Flesch 142 (1993) 89

Fracture criterion of a mixed-mode crack based on CED (Crack Energy Density), T. Utsunomiya and K. Watanabe 142 (1993) 101

Heavy-Section Steel Technology Program Overview, W.E. Pennell 142 (1993) 117

Risk assessment in setting inservice inspection priorities at Light Water Reactors, T.V. Vo, B.F. Gore, F.A. Simonen and S.R. Doctor 142 (1993) 239

Formulation of dynamic stability of fluid-filled shells, K. Tsukimori, W.K. Liu and R.A. Uras 142 (1993) 267

Dynamic stability characteristics of fluid-filled shells under multiple excitations, K. Tsukimori, W.K. Liu, Y.H. Tsao and R.A. Uras 142 (1993) 299

Materials properties

Elastic-plastic defect assessment based on ductile fracture process, Y.M. Dong, W. Yang and K.C. Hwang 142 (1993) 27

Heavy-Section Steel Technology Program: recent developments in crack initiation and arrest research, W.E. Pennell 142 (1993) 255

Mechanical design, component fabrication and testing

Heavy-Section Steel Technology Program Overview, W.E. Pennell 142 (1993) 117

Interfacing Systems LOCA (ISLOCA) component pressure capacity methodology and typical plant results, D.A. Wesley 142 (1993) 209

Reactor concepts and systems analysis

Design and calibration of leak detection systems by thermal hydraulics and fracture mechanics analyses, G. Bartholomé, W. Kastner and E. Keim 142 (1993) 1

Dependent failure analysis of NPP data bases, S.E. Cooper, E.V. Lofgren, P.K. Samanta and S.-M. Wong 142 (1993) 137

Incorporation of passive components aging into PRAs, J.H. Phillips, W.S. Roesener, H.L. Magleby and V. Geidl 142 (1993) 167

Research and development at the Structural Materials Information Center, C.B. Oland, D.J. Naus and E.G. Arndt 142 (1993) 179

Safeguard engineering

Design and calibration of leak detection systems by thermal hydraulics and fracture mechanics analyses, G. Bartholomé, W. Kastner and E. Keim 142 (1993) 1
Identification and assessment of containment and release management strategies, J.R. Lehner, C.C. Lin and P. Neogy 142 (1993) 189
Evaluation methodology for seismic base isolation of nuclear equipments, K. Ebisawa and T. Uga 142 (1993) 319

Structural design, construction methods and testing

Probabilistic methods for condition assessment and life prediction of concrete structures in nuclear power plants, B.R. Ellingwood and Y. Mori 142 (1993) 155

Radiation shielding

An overview of the ORNL/NRC program to address aging of concrete structures in nuclear power plants, D.J. Naus, C.B. Oland, B. Ellingwood, Y. Mori and E.G. Arndt 142 (1993) 327

Nuclear safety

Dependent failure analysis of NPP data bases, S.E. Cooper, E.V. Lofgren, P.K. Samanta and S.-M. Wong 142 (1993) 137
Probabilistic methods for condition assessment and life prediction of concrete structures in nuclear power plants, B.R. Ellingwood and Y. Mori 142 (1993) 155
Incorporation of passive components aging into PRAs, J.H. Phillips, W.S. Roesener, H.L. Magleby and V. Geidl 142 (1993) 167
Research and development at the Structural Materials Information Center, C.B. Oland, D.J. Naus and E.G. Arndt 142 (1993) 179
Interfacing Systems LOCA (ISLOCA) component pressure capacity methodology and typical plant results, D.A. Wesley 142 (1993) 209
U.S. NRC Accident Management Program, L.M. Shotkin 142 (1993) 225
Consideration of external events in the individual plant examination program, J.T. Chen 142 (1993) 231
Evaluation methodology for seismic base isolation of nuclear equipments, K. Ebisawa and T. Uga 142 (1993) 319
An overview of the ORNL/NRC program to address aging of concrete structures in nuclear power plants, D.J. Naus, C.B. Oland, B. Ellingwood, Y. Mori and E.G. Arndt 142 (1993) 327
Interfacial area density, mean radius and number density measurements in bubbly two-phase flow, S. Kalkach-Navarro, R.T. Lahey, Jr., D.A. Drew and R. Meyder 142 (1993) 341